

# Introduction of Possible Topics for the SPE #3

#### Herman Graves – U.S. Nuclear Regulator Commission Robert Dameron – David Evans & Associates Lili Akin – Sandia National Laboratory

### June 30, 2010



Sandia is a multiprogram laboratory operated by Sandia Corporation, a Lockheed Martin Company, for the United States Department of Energy's National Nuclear Security Administration under contract DE-AC04-94AL85000.



# **Objective**



# Primary Objective of Workshops:

- Enhance Knowledge in the Characterization of Prestressed Concrete Containment Vessels (PCCVs)
- Assess the Current Practices and State of the Art with Respect to the Calculation of Response to Severe Accident Conditions
- Improve Knowledge In:
  - Local Containment Behavior Under Beyond Design Basis Pressures
  - Characterization of Leakage Behavior as a Function of Pressure and Temperature
  - Probabilistic Aspects of Containment Response





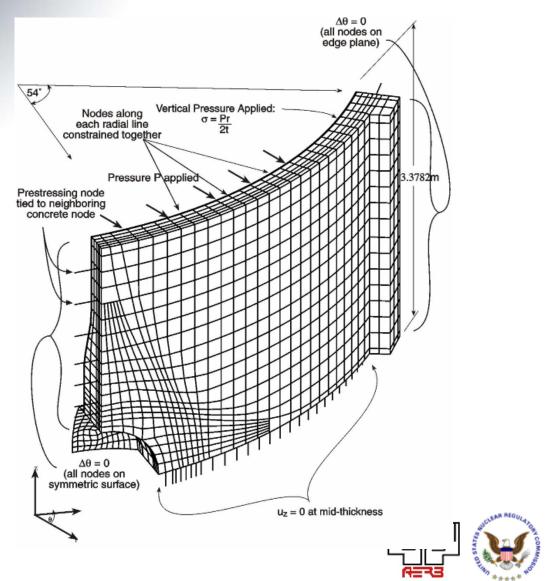
# **SPE and Related Analysis Work**

- An additional goal of this work can be to improve industry consensus on
  - means and methods for analyses
  - selection of FE model types, and modeling extent
  - applicable loads and problem definitions
- For example,
  - Some scopes and details of FE models....
  - Definitions of pressure plus thermal histories....





### **Choice of Models / Examples**

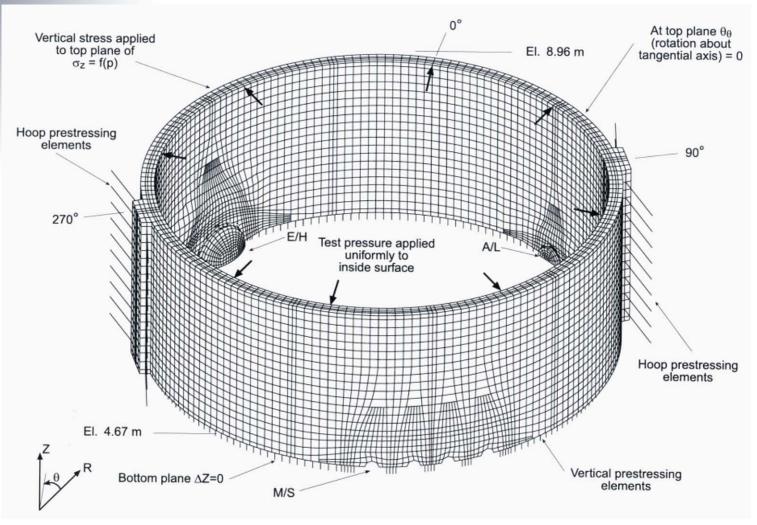




Vg# 4

NUCLEAR ENERGY & GLOBAL SECURIT

### **Choice of models / Examples**





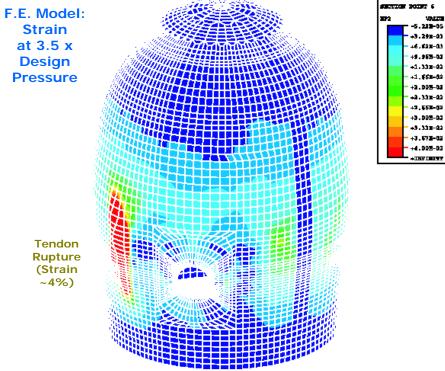


- NUCLEAR EVERGY & GLOBAL SECURITY
  - Stage 1 of Round Robin Analysis
    - Behavior of Tendon Forces as a Function of Containment Dilation
    - Friction Models Used to Represent Slippage of Prestressing Cables
    - Steel-Concrete Interaction Surrounding Penetrations
    - Failure Mechanisms in the Liner
    - Use of Nominal Design Parameters Versus In-Situ Material Properties and Boundary Conditions
  - Stage 2 of Round Robin Analysis
    - Characterization of Leakage Behavior of Containment Vessels as a Function of Pressure and Temperature



### Phase 1 – Uses 1:4 Scale PCCF Structural Failure Mode Test as Starting Point







NUCLEA

R/ENERGY

& GLOBAL

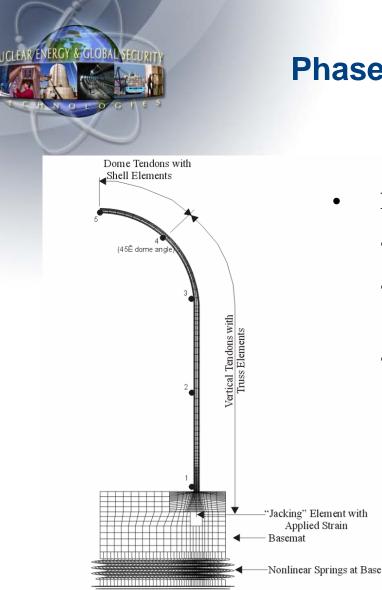
SECURIT



### Phase 1.1 Tendon Forces

- Observations from Previous Analyses Related to Hoop Tendon Measurements as a Function of Increasing Pressure:
  - Changes in Tendon Stress Distribution Under Loading
  - Interior Tendon Forces Exceeded Force at Anchor
  - Poor Agreement in Hoop Tendon Stress Distribution Despite Good Agreement With Radial Displacements
  - Overprediction of Dome and Overall Vertical Displacements and Anchor Forces
  - Underprediction of Interior Gage Stresses
- Participants Will Be Asked to Analytically Explore Tendon Forces as a Function of Containment Dilation





# Phase 1.1 Tendon Forces (continued)

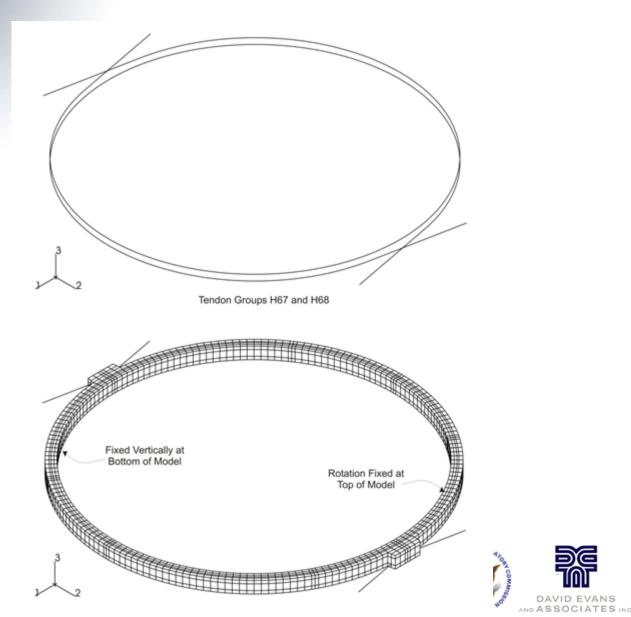
- Participants Shall Output:
  - Displacements (Positions 1-11)
  - Rebar Strains at Cylinder Midheight (Output Locations 22 and 23)
  - Liner Strains at Cylinder Midheight (Output Locations 38 and 39)





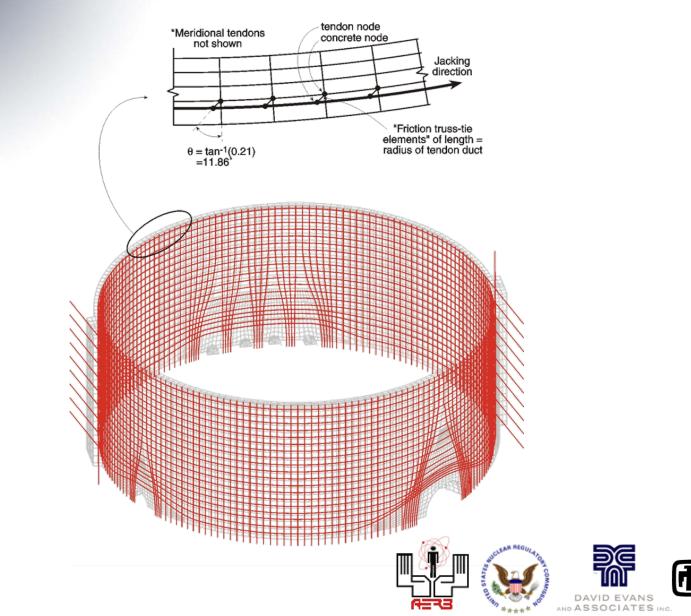
### **Phase 1.1 Tendon Forces (continued)**

Sandia National Laboratories





# **Phase 1.2 Slippage of Prestressing Cables**



Sandia National Laboratories

R/ENERGY

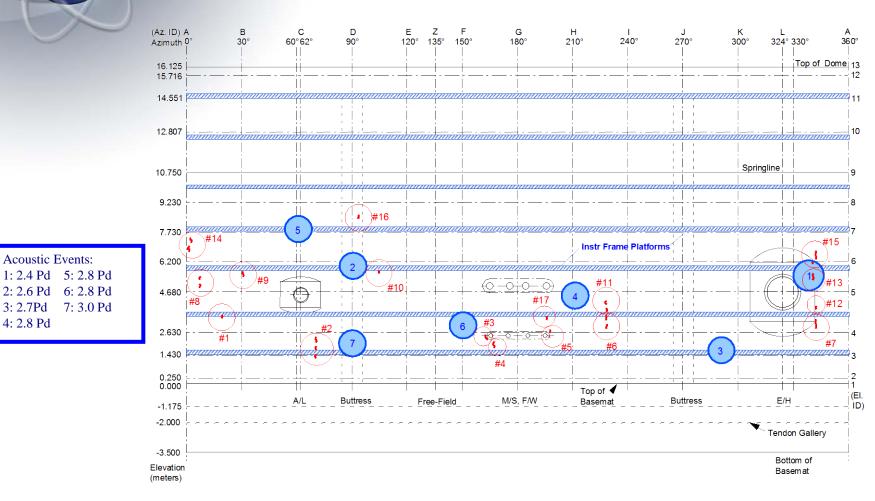
GLOBA

NUCLEA



## Phase 1.3 Steel-Concrete Interface

- Participants Shall Investigate the Separation Between the Concrete and Sleeves, the Stress Concentrations that Lead to Liner Tearing, and the Development of Potential Leak Paths Using Strain Information
  - Separations Were Observed Surrounding Containment Penetrations During Testing of the PCCV Structure
  - Many of the Highest Strains Recorded During the Limit State Test Were Near the Mainsteam and Feedwater Penetrations
  - Wide Variation in Peak Strain Data at Locations with Identical Geometry Due to Slight Variations in Liner Thickness (Caused by Manufacturing and Weld Repair Grinding), Gage Position Relative to the Collar/Weld, Material Properties, etc.
- Participants Shall Quantify the Risk Associated with the Formation of a Gap as a Function of a Potential Leak Path



# Phase 1.3 Steel-Concrete Interface (continued)

NUPEC/NRC 1:4 Scale PCCV Model Liner

Linier Tears and Acoustic Events After LST



RENERGY

NUCLE/

GLOB/



## **Phase 1.3 Steel-Concrete Interface** (continued)

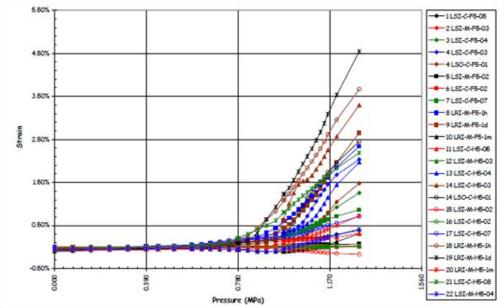








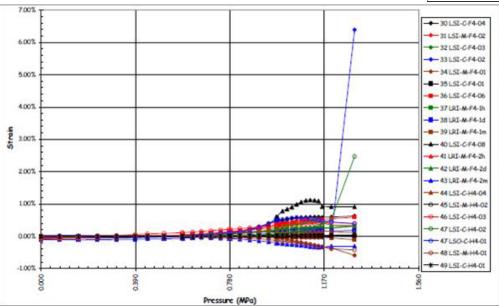
### **Phase 1.3 Steel-Concrete Interface**



#### Liner Strains (DOR) at M/S (Ref D-SN-P-220)



Liner Strains (DOR) at F/W (Ref D-SN-P-220)





# Phase 1.4 Failure Mechanisms

- Participants Shall Characterize the Liner Tearing Mechanism
  - Applicability of Fracture Mechanics Approach Versus Ductile Failure Approach in PCCV Model
  - Predict Tears in the Liner From Finite Element Model Strains
  - Compare with Map of Liner Tears





# Phase 1.5 Nominal Design vs. In-Situ Construction

- Participants Shall Evaluate Differences Between Model Design and In-Situ Construction
- Examples of Variation:
  - Concrete Material Properties (f'<sub>c</sub>, density, E, etc.)
  - Rebar Material Properties
  - Prestressing
  - Geometry
  - Temperatures Causing Localized High Strains
  - Etc.



## Phase 1.5 Nominal Design vs. In-Situ Construction (continued)

D2 CompletionF <sub>0</sub> <sup>-</sup> (MPa) Date F1 2-12-97 29.42 D1 D2 D2 D1 D2 D2 D1							= 29.42 M	i)	fc' = 44.13 MPa (6400 psi)				
Concrete Spe Pour Stri Completion For Date F1 2-12-97 29				Standa Cur		dard Field Curin		ing Standard Curing		Field Curing			
F3A 5-8-97 29 F3B 5-8-97 44	A 5-8-97 29.42 B 5-8-97 44.13 C4			D	ays	MPa (psi)		MPa (psi)		MPa (psi)		MPa (p	si)
F4 7-2-97 44.13   Planned Fo' (MPa)   Date Fo' (MPa)   C1 (11-98) 44.13   C2 (12-98) 44.13   C3 (1-99) 44.13   D1 (2-99) 44.13					7	32.06 (4650)		-		39.64 (5750)		-	
					14 35.37		(5130) -		42.89 (6220)		)	-	
D2 (4-99) 44 D3 (5-99) 44 F5 (5-00) 29 F6 (5-00) 44	D3 (6-99) 44.13 C2			28		38.96 (5650)		37.02 (5370)		48.61 (7050)		42.40 (61	150)
				ļ	91	-		39.16 (5680)		-		47.30 (68	860)
				104		47.85 (6940)		-		56.12 (8140)		-	
FI F5			fc' = 29.42 MPa			fc' = 44.			.13 MPa		1		
	Test Items	Standard Curing		Field Curing		Standard Curing		Field Curing					
	Compressive Strength (MPa)	(1 week)	33.64		29.42		39.13		40.99				
	Compressive Strength (MPa)	(4 weeks)	40.31		33.44		49.72		48.25				
	Compressive Strength (MPa)	(13 weeks)	51.39		41.68		60.21		48.84				
	Tensile Strength (MPa)	(13 weeks)	3.93		3.37		4.21		3.45				
	Flexural Strength (MPa) (13 weeks)		5.37		4.00		5.58		5.51				
	Young's Modulus (×103) (MPa)	oung's Modulus (×103) (MPa) (13 weeks)		29.03		27.95		31.97		26.97			
	Poisson's Ratio	(13 weeks)	0.20		0.1	8				0.18		A	Sandia National
Vg# 18	Density (ton/m3)	(13 weeks)	2.25		2.2			26	A BO	2.19 AVID EVANS		U	Laboratories

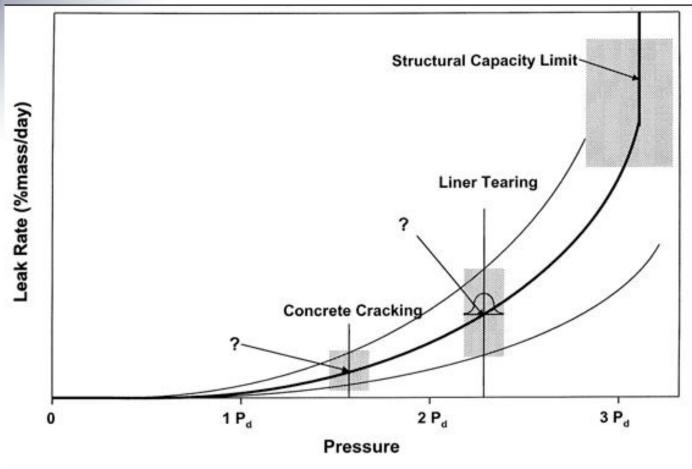
NUCLE

# Phase Two – Leakage as a Function of Pressure and Temperature

- Second Stage Focus: Characterization of Leakage Behavior as a Function of Pressure and Temperature
  - Leak Rate as a Function of Pressure
  - Leak Rate as a Function of Temperature and Pressure for Two Representative Cases
  - Prediction of Leak Rate as a Probabilistic Function of Pressure and Temperature in an Effort to Determine the Significance and Applicability of Standard Assumptions Used When Assessing Containment Structures
  - Recast Containment Analysis Conclusions Into a Containment Performance Framework



### Phase Two – Leakage as a Function of Pressure



**Consider Leak-Rate Versus Pressure Probabilistically** 



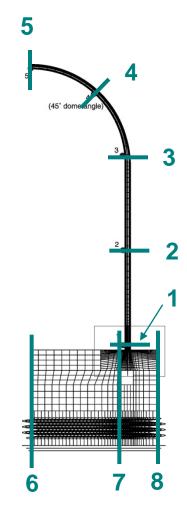
NUCLEA



# **Phase Two - Thermal Analysis**

- Model: Full-scale Axisymmetric with additional nodes throughout cylinder and dome - 12 throughthickness
- Material Properties: based on typical data
- Thermal Gradient calculation locations:
  - See figure
- Boundary Conditions:
  - Liner: Uniformly applied temperature; quasi-static, but transient
  - Dome & Cylinder: convection to air
  - Basemat/soil: conduction
- Reference:

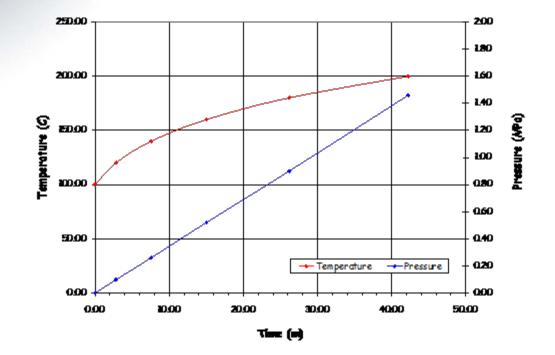
Dameron, et. al., "Analysis of Axisymmetric Presstressed Concrete Containment Vessel (PCCV) Including Thermal Effects", May, 2004









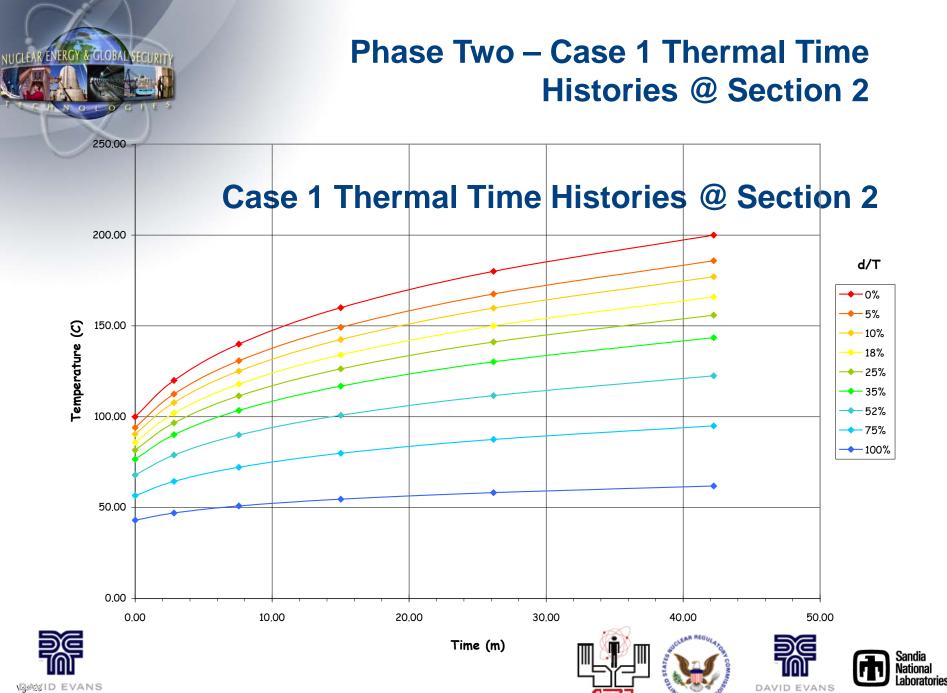


Case 1: A Saturated Steam Condition (Adding Temperature to Each Pressure Step from the Original PCCV Pressure Analysis)



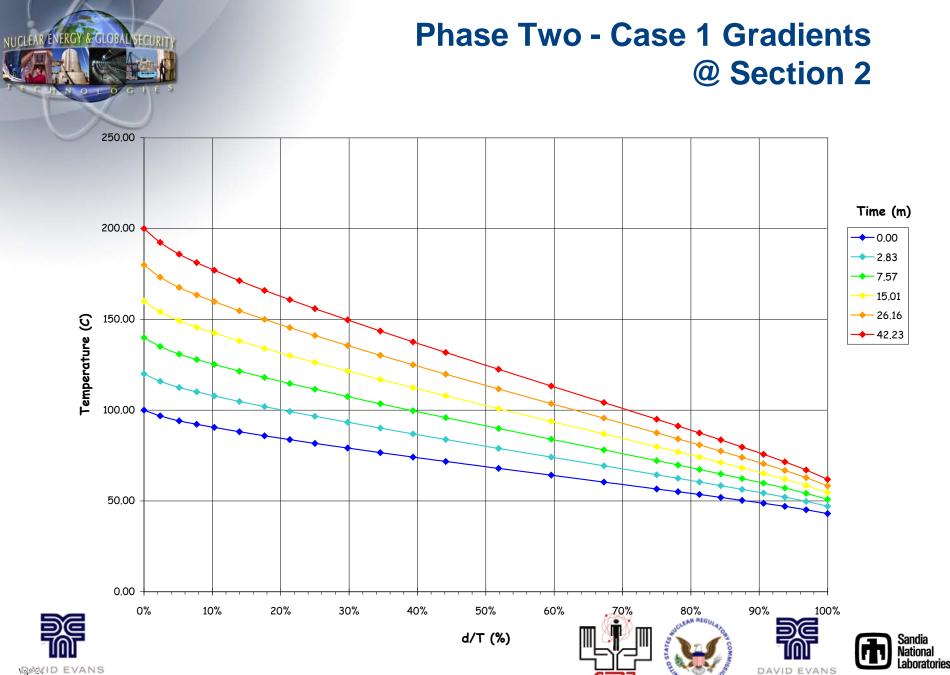
RENERGY

NUCLE/



AND ASSOCIATES INC.

AND ASSOCIATES INC.



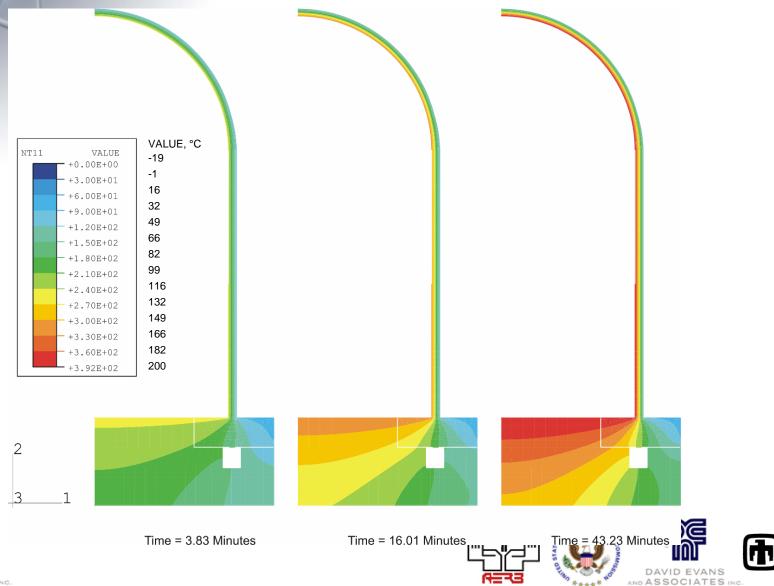
AND ASSOCIATES INC.

AND ASSOCIATES INC.



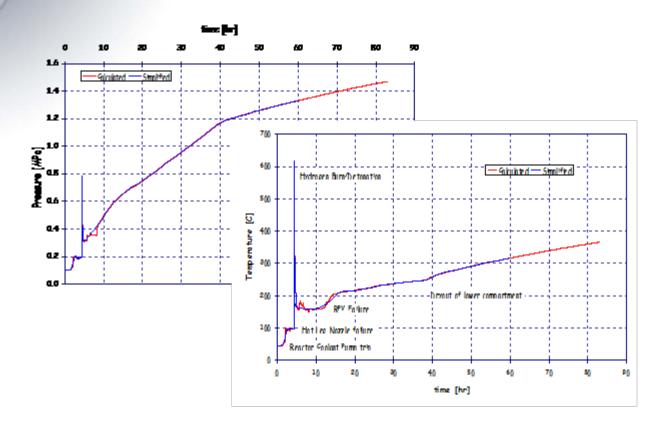
### **Phase Two - Case 1 Contours**

Sandia National Laboratories



DAVID EVANS

Phase Two – Leakage as a Function of Pressure and Temperature



Case 2: An Accident Safety Case (Essentially a Station Blackout Scenario with a Hydrogen Burn at About 4 <sup>1</sup>/<sub>2</sub> Hours Into the Event)

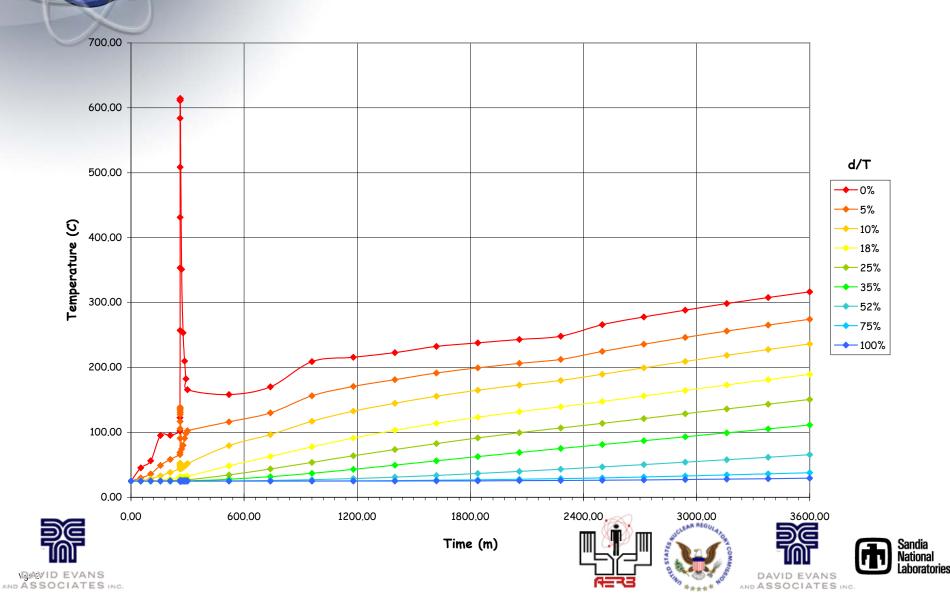


r energy

NUCLE/

NUCLEAR EVERGY & GLOBAL VECURITY

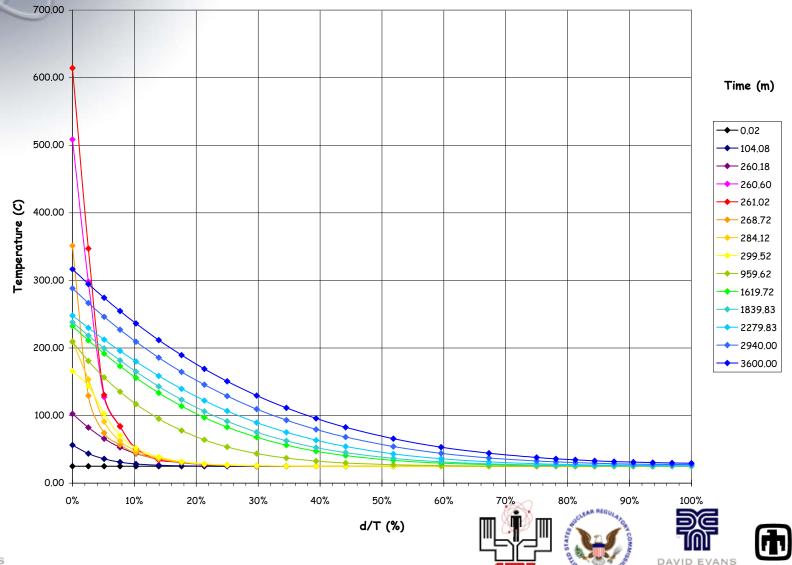
### Phase Two - Case 2 Thermal Time Histories @ Section 2



### Phase Two - Case 2 Gradients @ Section 2

Sandia National Laboratories

AND ASSOCIATES INC



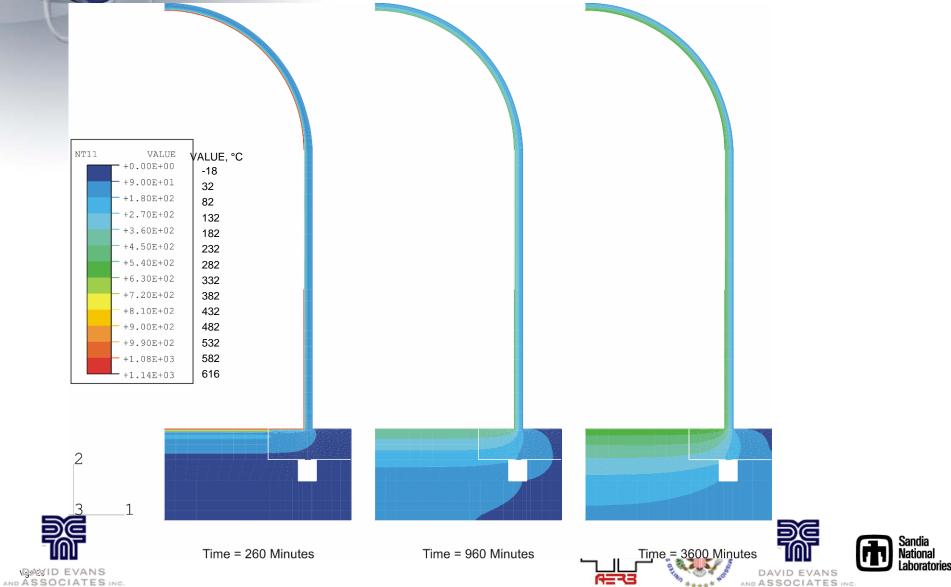
QAVID EVANS

NUCLEAR ENERGY

& GLOB/



### **Phase Two - Case 2 Contours**





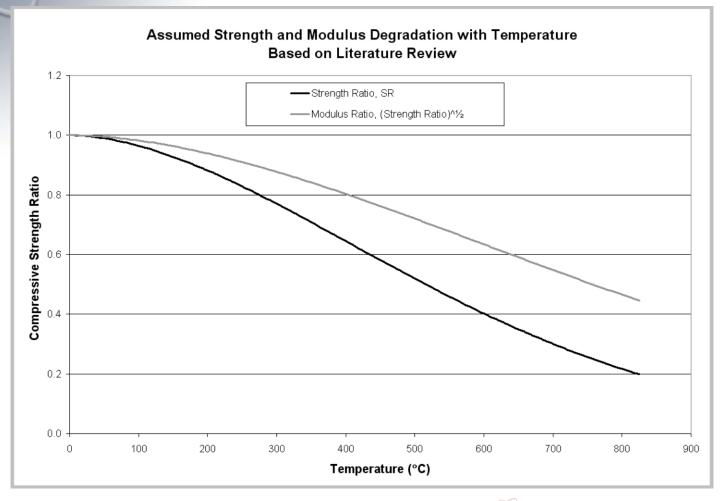
### **Phase Two – Additional Goal**

• Another goal is to reach consensus on influence of temperatures on containment material behaviors





# Phase Two - Concrete Degradation due to Temperature

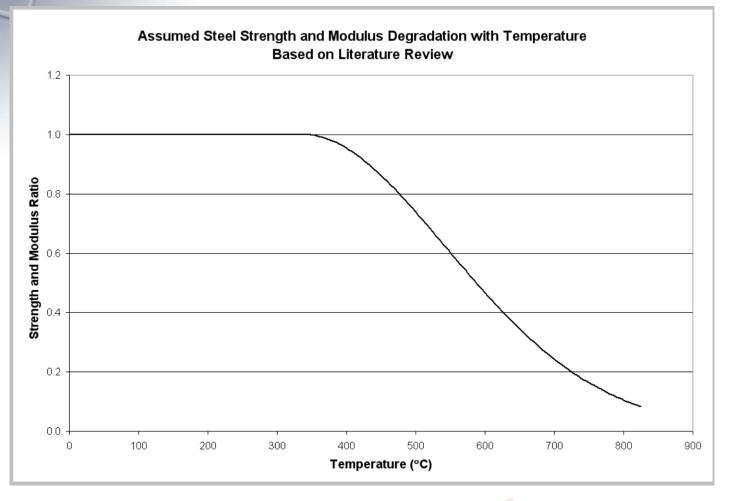








# Phase Two - Steel Degradation due to Temperature









# **Proposed Schedule**

- Initial Workshop June 30-July 2, 2010
- Phase One Calculations and Documentation Due October 22, 2010
- Phase Two Calculations and Documentation Due July 29, 2011
- Final Workshop (Location TBD) November 2011
- Final Report Sent to Participants February 2012

